

Linear Accelerator Direct Shielded Doors

An Approach for Calculating the Specialized Shielding Required Adjacent to the Door

Example Calculations

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Example Calculations are Based on NCRP Report No. 151

Page 2

- Report Title: "Structural Shielding Design and Evaluation for Megavoltage X- and Gamma-Ray Radiotherapy Facilities"

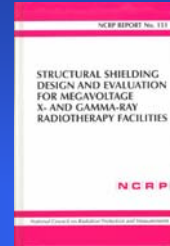
- Released December 31, 2005

- Calculations here illustrate the NCRP 151 recommendations

- Previous NCRP reports are also cited in some cases

- e.g., NCRP 51 and NCRP 79

- Detailed example calculations can be found at:
http://www.therapyphysics.com/WC2009_Examples.pdf



Detailed example calculations can be found on web site

Linear Accelerator Energy

Page 3

- BJR #11 megavoltage (MV) definition used here
 - British Journal of Radiology (BJR) Supplement No. 11
- Comparison of BJR #11 and BJR #17 MV definitions

BJR #11 MV	4	6	10	15	18	20	24
BJR #17 MV	4	6	10	16	23	25	30

NCRP 151 Recommended Workload [1 of 2]

Page 4

- Workload (W)
 - "Time integral of the absorbed-dose rate determined at the depth of the maximum absorbed dose, 1 m from the source"
- 450 Gy/wk maximum weekly workload cited in NCRP 151
 - Kleck (1994)
 - Maximum 350 Gy/wk for 6 MV
 - Maximum 250 Gy/wk at high MV for dual energy
 - Mechalacos (2004)
 - Maximum 450 Gy/wk for 6 MV single-energy
 - Maximum 400 Gy/wk for dual energy
 - NCRP 151 Section 7 examples assume 450 Gy/wk at high MV

450 Gy / wk absorbed dose is the default weekly workload

NCRP 151 Recommended Workload [2 of 2]

Page 5

- 30 patients treated per day is default assumption
 - NCRP 151 default recommendation for busy facility
 - Can also base on a conservative estimate influenced by factors such as historical workload and demographics
 - e.g. lower patient workload for facility in small town
- 3 Gy absorbed dose per patient treatment default
 - Assumption used in NCRP 151 Section 7 examples
 - Consistent with 450 Gy/wk with 30 patients treated per day
 - 450 Gy/wk = 5 treatments/wk/patient x 3 Gy/treatment x 30 patients
 - Equivalent to 219 cGy treatment fraction (0.73 tissue maximum ratio)
 - Intentionally somewhat conservative (compared to ~200 cGy fraction) since no specific allowance for quality or maintenance workload
 - Can be based on direct knowledge of accelerator use instead
 - But preferable to stick with the NCRP 151 default

450 Gy/wk is consistent with 30 patients & 3 Gy/treatment

Workload Assumptions for Dual Energy Linear Accelerators

Page 6

- Preferable to assume full 450 Gy/wk workload is at the higher energy
 - Simpler, more conservative calculation
 - Appropriate for new construction
- For existing construction, dual-energy calculation may be appropriate
 - If modifications to existing vault are difficult and size constrained
 - Split 30 patient workload to ensure at least 250 Gy/wk at higher MV
 - With 17 patients, 255 Gy/wk at higher MV

Mode	Gy/wk/patient	Patients/day	W (Gy/wk)
Single x-ray mode	15	30	450
Dual x-ray mode	15	30	450
High-X mode	15	17	255
Low-X mode	15	13	195

At least 250 Gy/wk at high MV mode

Radiation Protection Limits

Page 7

- Shielding Design Goal (P)
 - Level of dose equivalent (H) used in the design calculations
 - Applies to barriers designed to limit exposure to people
 - » Limiting exposure to unoccupied locations is not the goal
 - Stated in terms of mSv at the point of nearest occupancy
- NCRP 151 recommended values for shielding design goal
 - 0.10 mSv/week for controlled areas
 - 0.02 mSv/week for uncontrolled areas
- Lower values may be applicable in your country

Controlled Areas

Page 8

- Limited-access area in which the occupational exposure of personnel to radiation or radioactive material is under the supervision of an individual in charge of radiation protection
- Access, occupancy and working conditions are controlled for radiation protection purposes
- Areas are usually in the immediate areas where radiation is used, such as treatment rooms and control booths, or other areas that require control of access, occupancy, and working conditions for radiation protection purposes
- The workers in these areas are those individuals who are specifically trained in the use of ionizing radiation and whose radiation exposure is usually individually monitored

Uncontrolled Areas

Page 9

- All other areas in the hospital or clinic and the surrounding environs
- Trained radiation oncology personnel and other trained workers, as well as members of the public, frequent many areas near controlled areas such as examination rooms or restrooms
 - Choice of appropriate occupancy factors ensures the protection of both those who are occupationally exposed as well as others who might be exposed in these areas

Radiation Protection Limits for Locations

Page 10

- Protected location
 - Walls: 0.3 m beyond the barrier
 - Ceilings: 0.5 m above the floor of the room above the vault
 - Floors: 1.7 m above the floor of the room below
- Permissible dose at protected location depends on occupancy
- Occupancy factor (T):
 - Fraction of time a particular location may be occupied
- Maximum shielded dose rate at protected location: P/T
 - Assuming occupancy factor T for protected location

Max shielded dose rate traditionally referred to as P/T

NCRP 151 Recommended Occupancy

Page 11

- T=1: Areas occupied full-time by an individual) e.g. administrative or clerical offices; treatment planning areas, treatment control rooms, nurse stations, receptionist areas, attended waiting rooms, occupied space in nearby building
- T= 0.5: Adjacent treatment room, patient examination room adjacent to shielded vault
- T = 0.2: Corridors, employee lounges, staff rest rooms
- T = 0.125: Treatment vault doors
- T = 0.05: Public toilets, unattended vending rooms, storage areas, outdoor areas with seating, unattended waiting rooms, patient holding areas, attics, janitor's closets
- T = 0.025: Outdoor areas with only transient pedestrian or vehicular traffic, unattended parking

Occupancy Factor Selection

Page 12

- For interior locations, T=1 and T=0.2 are most common
 - T = 1 for work locations
 - T = 0.2 for locations not occupied continuously
- For exterior locations, T = 0.05 is most common
- T < 1 now appropriate for some controlled locations
 - Use with T = 0.125 for vault entrance with caution: any higher occupancy location further away must also be protected
 - T = 0.5 for adjacent vault appears to be reasonable assumption
- Select T = 0.05 for interior locations with caution
 - Should be very unlikely to be occupied (storage, attic, closets)
- T = 0.025 for exterior locations with restricted access
 - NRC hourly limit is more constraining for unrestricted locations

Page 13

Use Factor

- Use Factor (U) is the fraction of the workload for which the primary beam is directed at the barrier in question
- Traditionally U = 0.25 for lateral barriers, ceiling, & floor
- U = 0.1 for tapered portions of ceiling barrier (Example 11)
- Applies to primary barrier calculations, usually not secondary
- NCRP 151 Table 3.1 below consistent with these values
 - TBI may require deviation from these values

90° gantry angle intervals 45° gantry angle intervals

Angle Interval Center	U (percent)	Standard Deviation (percent)	Angle Interval Center	U (percent)	Standard Deviation (percent)
0° (down)	31.0	3.7	0° (down)	25.6	4.2
90° and 270°	21.3 (each)	4.7	45° and 315°	5.8 (each)	3.0
180° (up)	26.3	3.7	90° and 270°	15.9 (each)	5.6
			135° and 225°	4.0 (each)	3.3
			180° (up)	23.0	4.4

Page 14

Secondary Barrier Photon Leakage

- Leakage unshielded dose rate

$$H_L = \frac{W \times \text{leakage fraction}}{d_{sec}^2}$$
 - Assumes H_L in Sv and W in Gy
 - 0.1% leakage fraction is customary
 - Secondary distance d_{sec} in meters
- Calculate shielded dose rate using TVLs in NCRP 151 Table B.7
- Calculation tends to be conservative
 - Typical leakage 5X or more lower than 0.1% requirement
 - Unlike primary barriers, generally no need for extra margin

Page 15

Table 2: Leakage TVLs (mm)

Linac MV	Lead		Concrete		Steel		Earth		Borated Poly	
	TVL1	TVLe	TVL1	TVLe	TVL1	TVLe	TVL1	TVLe	TVL1	TVLe
4	57	57	330	280	96	96	517	439	817	693
6	57	57	340	290	96	96	533	455	842	718
10	57	57	350	310	96	96	549	486	866	767
15	57	57	360	330	96	96	564	517	891	817
18	57	57	360	340	96	96	564	533	891	842
20	57	57	360	340	96	96	564	533	891	842
25	57	57	370	350	96	96	580	549	916	866

NCRP 151 Primary TVL Table B.2 NCRP 151 Table B.7 Varian TVL ratio relative concrete Est. by density vs. concrete
 concrete = 2.35 g/cm³ [NCRP 151, p. 69] earth density = 1.6 g/cm³ [NCRP 151, p. 72] BPE = 0.95 g/cm³ [NCRP 151, p. 162]

Note: NCRP 51 Figure E.14 indicates lead TVL is maximum near 6 MeV, so using primary TVL for leakage is reasonable.
 No data in NCRP 151 for steel leakage TVL.
 NCRP 51 Figure E.13 implies steel leakage TVL should be less than primary.
 Rationale for 96 mm steel TVL based on Varian document #12004 on next chart.

Page 16

Intensity Modulated Radiation Therapy (IMRT)

- IMRT requires increased monitor units per cGy at isocenter
 - IMRT ratio is the ratio of MU with IMRT per cGy at isocenter
- Percent workload with IMRT impacts shielding
 - 50% typically assumed; 100% if vault is dedicated to IMRT
- Account for IMRT by multiplying workload by IMRT factor
 - IMRT Factor = % IMRT x IMRT ratio + (1 - % IMRT)
- Leakage Workload: $W_L = W \times \text{IMRT Factor}$
 - W_L replaces W in leakage unshielded dose calculation with IMRT
- Lower IMRT factor appropriate for neutrons if calculate shielding at the higher MV for a dual MV machine

Page 17

IMRT Ratio Typical Values

Manufacturer	IMRT Ratio	Percent IMRT	IMRT Factor	
			Photon	Neutron
Varian	3	50%	2	1
Siemens	5	50%	3	1.5
NOMOS	10	50%	5.5	2.75
Tomotherapy	16	100%	16	NA

- Typically assume 50% of treatments with IMRT
 - Pessimistic assumption for dual energy machine since most IMRT done at lower energy (e.g., >75% at 6 MV, <25% at 18 MV)
- Neutron IMRT factor (applicable to dual energy) assumes IMRT equally at high and low energy
 - Since most IMRT is done at the lower energy, an even lower neutron IMRT factor may be appropriate

Page 18

Neutron Leakage Fraction

- Neutron leakage unshielded dose rate

$$H_n = \frac{W_n \times H_0}{d_{sec}^2}$$
 - H_n in Sv and W_n in Gy
 - H_0 is neutron leakage dose equivalent fraction normalized to 1 m from target
- H_0 in Table 4 normalized to 1 m
 - Varian* and Siemens** values based on manufacturer data
 - Elekta data from Site Planning Guide***
 - GE data based on NCRP 151 Table B.9 normalized to 1 m

Vendor	MV	H_0 @ 1 m Sv/Gy
Varian	10	4.0E-05
	15	7.0E-04
	18	1.5E-03
	20	1.9E-03
Siemens	10	2.0E-05
	15	4.2E-04
	18	9.9E-04
	20	1.4E-03
Elekta / Philips	10	3.0E-04
	15	7.0E-04
	18	1.5E-03
	20	2.0E-03
GE	12	1.8E-04
	15	6.4E-04
	25	2.7E-03

* Varian: <http://www.varian.com/osup/pdf/12000.pdf> [Page 12, Average of 4 positions]
 ** Siemens: Conservative neutron leakage dose rates in patient plane with Q=10
 *** Elekta: Nisy Ipe, "Neutron Shielding Design and Evaluations", 2007 AAPM Summer School

Neutron Leakage TVL

Page 19

- TVL recommendation based on NCRP 79
 - TVL_{nc} = 155 + 56 * Neutron MV for concrete
 - 211 mm at 1 MV is traditional neutron leakage TVL for concrete
 - TVL_{nc} = 62 + 34 * Neutron MV for borated polyethylene (BPE)
 - 96 mm at 1 MV is traditional neutron leakage TVL for BPE
 - Estimate other material from concrete or BPE based on hydrogen content
 - Lead and steel provide negligible neutron attenuation

MV	Concrete		Earth		Borated Poly		Application
	TVL1	TVLe	TVL1	TVLe	TVL1	TVLe	
1	211	211	331	331	96	96	Leakage

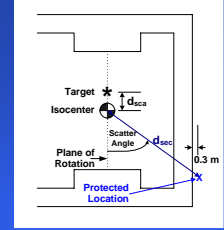
Secondary Barrier Patient Scatter

Page 20

- Patient scatter unshielded dose rate

$$H_{ps} = \frac{aWU(F/400)}{d_{sc}^2 d_{sc}^2}$$

- a = scatter fraction for 20 x 20 cm
- F is maximum field area in cm²
 - NCRP 151 examples use F=1600 (conservative 40x40 cm field)
- Effective F is smaller with IMRT
 - F=225 cm² w/ IMRT (15 x 15 cm)
- F = (1-% IMRT) x 1600 + % IMRT x 225
- Typically use F=1600 even if IMRT is used to add conservatism
 - Safety survey done w/o IMRT
 - IMRT seldom used at higher MV for dual energy machines
 - Primary beam adds to patient scatter at small scatter angles



- Scatter fraction as function of MV and scatter angle in NCRP 151 Table 5.4
- Scatter energy as function of MV and scatter angle in NCRP 151 Table B.6

Use Factor (U) and Scatter

Page 21

- Use Factor is typically taken as 1 for secondary calculations
 - Invariably true for leakage calculations
- Scatter is significant only for secondary barriers immediately adjacent to primary barriers
 - Scatter is negligible for all other orientations
 - NCRP 151: "However, if the [scatter] calculation is performed with the minimum angle of scatter from the patient to the point of calculation and a use factor of 1 is also used, the barrier thickness will be overestimated due to the conservatively higher scatter fraction from the smaller scattering angles"
- Sometimes appropriate to apply use factor to scatter
 - U = 0.25 may be appropriate if scatter angle < 35°
 - i.e., secondary barrier immediately adjacent to primary barrier
 - U=0.25 best used only for retrofit (to avoid unnecessary modifications) or if there are severe space constraints
 - Otherwise U = 1

NCRP 151 Table B.4 Patient Scatter Fraction for 400 cm² Field

Page 22

- Scatter fraction increases as angle decreases
- Scatter fraction vs MV may increase or decrease
 - Tends to increase with MV at small scatter angles
 - Decreases with increasing MV at large scatter angles

Linac MV	Angle (degrees)							
	10	20	30	45	60	90	135	150
4	1.04E-02	6.73E-03	2.77E-03	2.09E-03	1.24E-03	6.39E-04	4.50E-04	4.31E-04
6	1.04E-02	6.73E-03	2.77E-03	1.39E-03	8.24E-04	4.26E-04	3.00E-04	2.87E-04
10	1.66E-02	5.79E-03	3.18E-03	1.35E-03	7.46E-04	3.81E-04	3.02E-04	2.74E-04
15	1.51E-02	5.54E-03	2.77E-03	1.05E-03	5.45E-04	2.61E-04	1.91E-04	1.78E-04
18	1.42E-02	5.39E-03	2.53E-03	8.64E-04	4.24E-04	1.89E-04	1.24E-04	1.20E-04
20	1.52E-02	5.66E-03	2.59E-03	8.54E-04	4.19E-04	1.85E-04	1.23E-04	1.18E-04
24	1.73E-02	6.19E-03	2.71E-03	8.35E-04	3.91E-04	1.76E-04	1.21E-04	1.14E-04

Scatter Tenth Value Layers

Page 23

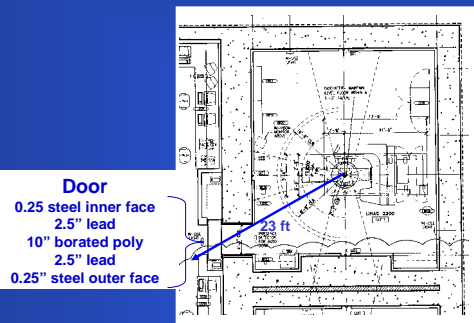
- From NCRP 151 Table B.5 and Figure A.1

MV	Concrete Scatter TVL (mm)						Recommended Steel Scatter TVL (mm)					
	Scatter Angle (degrees)						Scatter Angle (degrees)					
	15	30	45	60	90	20	30	45	60	75	90	
4	270	250	240	220	180	4	78	72	68	63	58	50
6	280	260	240	220	190	6	83	78	70	64	58	53
10	300	270	250	230	190	10	84	78	71	65	58	54
15	320	280	250	230	210	15	89	82	74	67	62	59
18	330	280	260	230	210	18	92	84	76	68	62	61
20	340	290	260	240	210	20	93	86	78	70	63	61
24	350	300	270	250	210	24	95	88	81	73	66	61

MV	Lead Scatter tenth-value layers (mm)															
	30				45				75				90			
	TVL1	TVLe	TVL1	TVLe	TVL1	TVLe	TVL1	TVLe	TVL1	TVLe	TVL1	TVLe	TVL1	TVLe		
4	46	46	33	37	24	31	18	25	13	19	9	13				
6	50	50	38	44	28	34	19	26	14	19	10	15				
10	51	51	43	45	31	36	21	27	15	19	12	16				
15	54	54	50	50	41	41	31	31	24	24	21	21				
18	55	55	51	51	43	43	32	32	24	24	22	22				
20	56	56	52	52	45	45	34	34	26	26	22	22				
25	57	57	54	54	48	48	39	39	29	29	22	22				

Example 1: Direct Shielded Door — Door Thickness

Page 24



Example 1: Direct Shielded Door Thickness Calculation [1 of 2]

Page 25

Line	Parameter	Units	Value	Calculation
a	Design Dose Limit (P)	mSv/wk	0.1	
b	Occupancy Factor (T)		1	
c	P/T	mSv/wk	0.100	a / b
d	Machine X-ray Energy	MV	18	
e	Vendor		Varian	

Line	Parameter	Units	Value		Calculation
			w/o IMRT	with IMRT	
a	Max Field Size	cm	40	40	
b	Fraction of Workload		50%	50%	
c	Effective Field Area	cm ²	1600.0		b ₁ a ₁ ² + b ₂ a ₂ ²
d	Effective Field Size	cm	40.0		sqrt (c)
e	Scatter Angle	deg	60		
f	Machine X-ray Energy	MV	18		
g	Scatter / 400 cm ²		4.24E-04		Function of e & f
h	Scatter Fraction		0.00170		g * c / 400

Example 1: Direct Shielded Door Thickness Calculation [2 of 2]

Page 26

Line	Parameter	Units	Photon Leakage	Photon Scatter	Neutron Leakage	Calculation
a	Workload / Treatment	Gy/pt	3	3	3	NCRP 151 default
b	Patients per Day	pt/day	30	30	30	NCRP 151 default
c	Workload (W)	Gy/wk	450	450	450	5 * a * b
d	Use Factor	Ratio	1	1	1	
e	Fraction		1.00E-03	1.70E-03	1.5E-03	18 MV values
f	IMRT Factor		2	1	1	
g	Isocenter to Protected Point Distance	ft	23.0	23.0	23.0	
h		m	7.0	7.0	7.0	g * 0.3048
i	Unshielded Dose Rate	mSv/wk	1.83E+01	1.55E+01	1.37E+01	1000*c*d*e ² /f ²
j	Transmission		8.44E-04	8.44E-07	8.81E-04	see below
k	Shielded Dose Rate	mSv/wk	0.0154	0.0000	0.0121	i * j
L	Total Shielded Dose Rate	mSv/wk		0.0276		Sum row k

Barrier	Material	Thickness	Photon Leakage			Scatter		Neutron			
			TVL (mm)	TVL (mm)	Trans.	TVL (mm)	TVL (mm)	Trans.	TVL (mm)	Trans.	
Inside Layer	0.25	7	Steel	96	96	8.39E-01	68	68	7.80E-01	N/A	1.00E+00
Layer #2	2.5	73	Lead	57	57	5.17E-02	32	32	5.11E-03	N/A	1.00E+00
Layer #3	10	293	Borated Poly	891	842	4.48E-01	230	230	5.31E-02	96	8.81E-04
Layer #4	2.5	73	Lead	57	57	5.17E-02	32	32	5.11E-03	N/A	1.00E+00
Outside Layer	0.25	7	Steel	96	96	8.39E-01	68	68	7.80E-01	N/A	1.00E+00
Slant Angle (degrees):	30		18 MV			Total:	8.44E-04	Total:	8.44E-07	Total:	8.81E-04

Direct-Shielded Door: Far Side of Entrance

Page 27

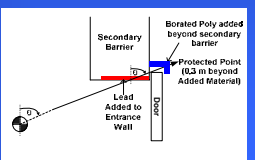
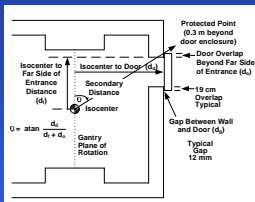
- Direct leakage path occurs at the far lateral edge of the door
- Slant thickness through the wall at the corner (d_c) is given by

$$d_c = \frac{d_o}{\cos(\theta)} - \frac{d_g}{\sin(\theta)} \quad \theta = \text{atan} \frac{d_i}{d_i + d_o}$$

- d_o is the door overlap
- d_g is gap between the wall & door

Shielding next to door

- Compensates for slant thickness (d_c) being inadequate for direct leakage and scatter
- Lead provides shielding needed for x-ray leakage & patient scatter
- Borated polyethylene provides shielding for neutron leakage



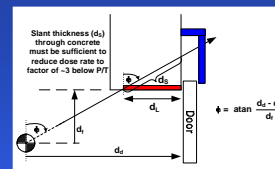
Length of Lead Required at Far Side of Entrance

Page 28

- Slant thickness (d_s) through the wall not benefiting from additional shielding

$$d_s = \frac{d_L}{\cos(\phi)}$$

$$\phi = \text{atan} \frac{d_i - d_L}{d_i}$$



- d_L is the length of the lead added to the wall
- d_g is gap between the wall & door
- d_i is isocenter to far side of entrance distance (in plane of rotation)
- d_g is isocenter to door distance (perpendicular to plane of rotation)

- Select d_L sufficiently large so that slant distance d_s through wall provides adequate shielding
 - Preferably factor of 3 margin to allow for scatter from door & HVAC penetration

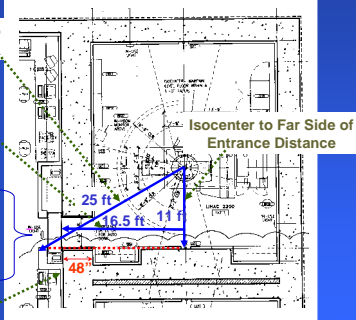
Example 2: Direct Shielded Door — Additional Shielding for Far Side of Door

Page 29

Secondary Distance to Far Side of Entrance

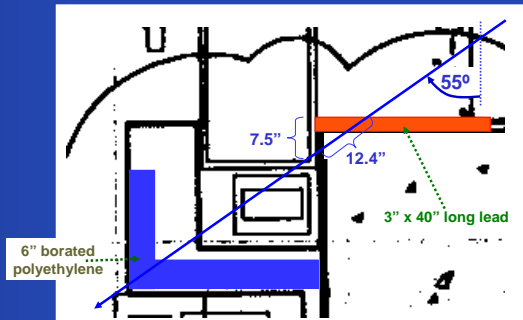
Isocenter to Inside Face of Door Distance

Secondary Barrier Thickness on Far Side of Entrance



Example 2: Direct Shielded Door — Additional Shielding for Far Side of Door

Page 30



Page 31

Example 2: Direct Shielded Door — Additional Shielding for Far Side of Door

48" concrete slant thickness is same as recommended lateral barrier thickness

3' x 32' long lead

6" borated polyethylene

PRESENCE DETECTOR FOR AUTO-DOOR

IN-USE LIGHT

Page 32

Example 2: Direct Shielded Door Far Side of Entrance Shielded Dose Rate Calculation [1 of 3]

Far Side of Door Distance, Thickness, and Length Calculations

Line	Parameter	Units	Value	Calculation
a	Door Overlap	in	7.5	
b	Gap Between Barrier and Door	in	0.5	
c	Distance from Isocenter to Far Side of Entrance	ft	11	
d	Distance from Isocenter to Inside Face of Door	in	132	c * 12
e	Distance from Isocenter to Inside Face of Door	ft	16.5	
f	Slant Angle at Far Side of Entrance	deg	54.8	atan(f / (a + d))
g	Slant Thickness at Corner	in	12.4	a / cos(g) - b / sin(g)
h	Slant Thickness through Lead	mm	315	25.4 * h
i	Thickness of Lead Added to Wall	in	3	Selected value
k	Slant Thickness through Concrete	mm	132	25.4 * j / cos(g)
L	Concrete Thickness	in	183	i - k
m	Concrete Thickness	in	4.15	L * cos(g) / 25.4
n	Borated Poly Thickness	in	6	Selected value
o	Borated Poly Slant Thickness	mm	186	25.4 * n / sin(g)
p	Minimum Desired Slant Thickness	in	42	Dose rate < P/T / 3
q	Minimum Length of Added Lead	in	35	p * sin(g)

- Thicknesses selected to make shielded dose rate less than dose limit

Page 33

Example 2: Direct Shielded Door Far Side of Entrance Shielded Dose Rate Calculation [2 of 3]

Far Side of Door Scatter Fraction Calculation

Line	Parameter	Units	Value		Calculation
			w/o IMRT	with IMRT	
a	Max Field Size	cm	40	40	
b	Fraction of Workload		50%	50%	
c	Effective Field Area	cm ²	1600.0		b ₁ *a ₁ ² + b ₂ *a ₂ ²
d	Effective Field Size	cm	40.0		sqrt(c)
e	Scatter Angle	deg	54.8		
f	Machine X-ray Energy	MV	18		
g	Scatter / 400 cm ²		5.42E-04		Function of e & f
h	Scatter Fraction		0.00217		g * c / 400

Page 34

Example 2: Direct Shielded Door Far Side of Entrance Shielded Dose Rate Calculation [3 of 3]

Line	Parameter	Units	Photon Leakage	Photon Scatter	Neutron Leakage	Calculation
a	Workload / Treatment	Gy/pt	3	3	3	NCRP 151 default
b	Patients per Day	pt/day	30	30	30	NCRP 151 default
c	Workload (W)	Gy/wk	450	450	450	5 * a * b
d	Use Factor	Ratio	1	1	1	
e	Fraction		1.00E-03	2.17E-03	1.50E-03	18 MV values
f	IMRT Factor		2	1	1	
g	Isocenter to Protected Point Distance	ft	25.0	25.0	25.0	
h	Unshielded Dose	mSv/wk	1.55E+01	1.68E+01	1.16E+01	g * 0.3048
i	Transmission		8.31E-04	6.14E-06	1.55E-03	see below
j	Shielded Dose	mSv/wk	0.013	0.000	0.018	i * j
L	Total Shielded Dose	mSv/wk		0.031		Sum row k

Barrier	Material	Thickness (mm)	Photon Leakage		Scatter		Neutron			
			TVL1 (mm)	TVL2 (mm)	Trans.	TVL (mm)	Trans.	TVL (mm)	Trans.	
Inside Layer	Lead	132	57	57	4.77E-03	36	36	2.01E-04	NA	1.00E+00
Layer #2	Concrete	183	360	340	2.90E-01	244	244	1.78E-01	211	1.36E-01
Layer #3	Borated Poly	186	891	842	6.01E-01	244	244	1.72E-01	96	1.14E-02
Layer #4					1.00E+00			1.00E+00		1.00E+00
Outside Layer					1.00E+00			1.00E+00		1.00E+00
Slant Angle (degrees): 54.8			18 MV	Total:	8.31E-04	Total:	6.14E-06	Total:	1.55E-03	

Page 35

Direct-Shielded Door: Near Side of Entrance Wall Scatter

- Geometry similar to short maze
 - Scatter is significant only if < 10 MV
- Unshielded dose rate

$$f H_s = f \frac{W U \alpha_0 A_0 \alpha_z A_z}{d_n^2 d_r^2 d_z^2}$$

where

- f = patient transmission (0.25)
- α₀ = first reflection coefficient
 - » NCRP 151 Table B.8a vs. MV
 - » 75° angle of reflection typical
- A₀ = beam area (m²) at wall
- α_z = 2nd reflection coefficient
 - » 0.5 MV at 75° in Table B.8a
- A_z = Maze cross section (m²)
 - » w_M x maze height

Page 36

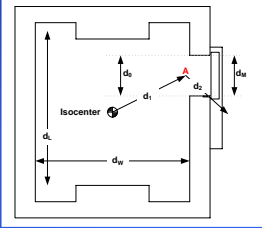
Direct-Shielded Door: Near Side of Entrance Neutrons / Capture Gammas

- Geometry similar to short maze
 - Therefore reasonable to apply maze calculation
- Geometry is not exactly the same as a short maze
 - Also reasonable to include margin since maze calculation may underestimate neutron dose rate
- Neutrons & capture gammas dominate scatter for MV ≥ 10
- Requires less material than far side of entrance
 - Lower unshielded dose rate
 - Lower energy

Page 37

Steps in Maze Neutron and Capture Gamma Calculation

- First step: Calculate neutron fluence at point A
- Second step: Calculate unshielded capture gamma dose rate at protected location
 - Uses neutron fluence at point A
- Third step: Calculate unshielded neutron dose-equivalent rate at protected location
 - Uses neutron fluence at point A
- Fourth step: Calculate attenuation of maze neutrons & capture gammas by the shielding at the near side of the entrance



Page 38

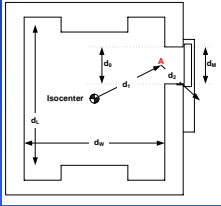
Neutron Fluence Calculation

- Neutrons / m² / Gy workload

$$\phi_A = \frac{\beta Q_n}{4 \pi d_1^2} + \frac{5.4 \beta Q_n}{2 \pi S_r} + \frac{1.3 Q_n}{2 \pi S_r}$$

- 1st term: Direct neutrons
- 2nd term: Scattered neutrons
- 3rd Term: Thermal neutrons

- where
 - β = head shielding transmission factor = 1.0 for lead, 0.85 for tungsten
 - d_1 = Distance from isocenter to point A
 - Q_n = Neutron source strength (NCRP 151 Table B.9)
 - S_r = Treatment room surface area (m²)
 - $S_r = 2(d_L d_w + h d_L + h d_w)$ where h is vault height



Page 39

NCRP 151 Table B.9 Total Neutron Source Strength (Q_n)

Vendor	MV	Q _n N/GY
Varian	10	6.0E+10
	15	7.6E+11
	18	9.6E+11
	20	9.6E+11
Siemens	24	7.7E+11
	10	8.0E+10
	15	2.0E+11
	18	8.8E+11
Elekta / Philips	20	9.2E+11
	24	1.5E+12
	10	1.4E+11
	15	3.2E+11
GE	18	6.9E+11
	20	9.6E+11
	24	1.4E+12
	12	2.4E+11
	15	4.7E+11
	18	1.5E+12
	25	2.4E+12

Page 40

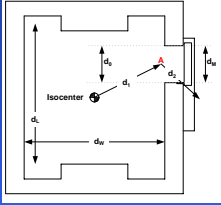
Capture Gamma Unshielded Dose Rate Calculation

- Capture gamma dose at protected location per workload at isocenter (Sv/Gy)

$$h_\phi = K \phi_A 10^{(-d_2 / TVD)}$$

- where
 - K = ratio of capture gamma dose at point A to neutron fluence = 6.9 x 10⁻¹⁶ m² Sv / neutron
 - d_2 = distance from point A to door
 - TVD = tenth-value distance (m) = 5.4 for 18-24 MV, 3.9 for 15 MV
- Weekly capture gamma dose rate

$$H_{cg} = W_{Ln} h_\phi$$
 - W_{Ln} is neutron leakage workload



Page 41

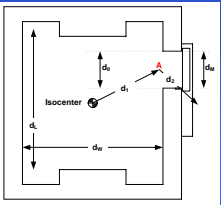
Neutron Unshielded Dose Rate Calculation

- Maze neutron dose-equivalent at protected location per neutron leakage workload at isocenter (Sv/Gy)

$$H_{n,D} = 2.4 \times 10^{-15} \phi_A \left[\frac{S_0}{S} \right]^{1/2} \left[1.64 \times 10^{(-d_2 / 1.9)} + 10^{(-d_2 / TVD)} \right]$$

- S_0 / S = ratio of inner maze entrance cross-section area ($S_0 = d_0 h$) to maze cross-section area ($S = d_m h$)
- d_2 = distance from point A to door
- TVD = tenth-value dist. = 2.06 S^{1/2}
- Maze neutron dose-equivalent

$$H_n = W_{Ln} H_{n,D}$$
- Add a factor of 5 margin
 - Since this particular geometry is only similar to a maze, not an exact match



Page 42

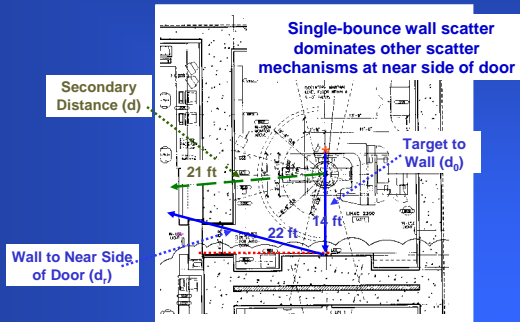
Short Maze Tenth Value Layers (TVLs)

- Capture Gamma TVL
 - NCRP 151: "for very short mazes ... a lead TVL of 6.1 cm may be required"
 - NCRP 151: "can range as high as 10 MeV" for very short mazes
 - Use primary 10 MV TVLs for material other than lead
- Neutron TVL
 - "maze door shielding, a conservatively safe recommendation is that a TVL of 4.5 cm be used in calculating the borated polyethylene (BPE) thickness requirement" [NCRP 151 p. 46]
 - "the average neutron energy at the maze entrance is reported to be ~100 keV" [NCRP 151 p. 46]
 - NCRP 79 TVL_n for concrete with 0.1 MV neutron energy: TVL_n = 155 mm + (56 mm/MV) * 0.1 MV = 161 mm

	Lead		Concrete		Steel		Borated Poly	
	TVL 1	TVL eq	TVL 1	TVL eq	TVL 1	TVL eq	TVL 1	TVL eq
Capture Gamma	61	61	410	370	110	110	1015	916
Neutron	N/A	N/A	161	161	N/A	N/A	45	45

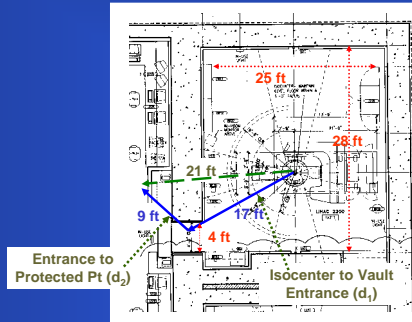
Example 3: Direct Shielded Door— Additional Shielding for Near Side of Door (Wall Scatter)

Page 43



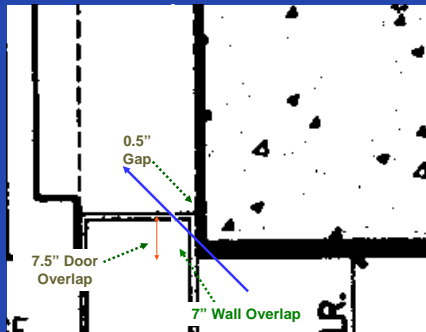
Example 3: Direct Shielded Door— Additional Shielding for Near Side of Door (Neutrons / Capture Gammas)

Page 44



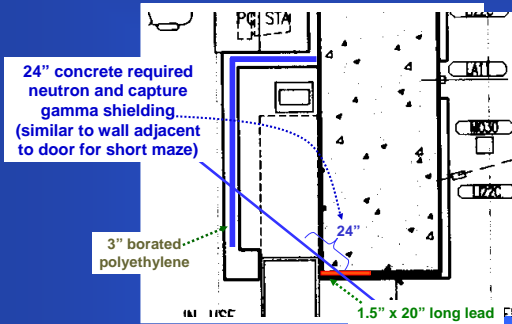
Example 3: Direct Shielded Door — Gap Between Wall and Door

Page 45



Example 3: Direct Shielded Door — Additional Shielding for Near Side of Door

Page 46



Example 3: Direct Shielded Door Near Side of Entrance Shielded Dose Rate Calculation [1 of 9]

Page 47

Near Side of Door Material Thickness Calculation				
Line	Parameter	Units	Value	Calculation
a	Door Overlap	in	7.5	
b	Gap Between Barrier and Door	in	0.5	
c	Angle at Near Side Wall	deg	45.0	
d	Wall Overlap Beyond Entrance	in	7.0	$(a \cdot \tan(c) - b) / \tan(c)$
e	Thickness of Lead Added to Wall	in	1.5	
f	Remaining Concrete Wall	in	5.5	$d - e$
g	Material Added beyond Wall	in	3	

- Material added to wall selected as required to make shielded dose rate less than dose limit

Example 3: Direct Shielded Door Near Side of Entrance Shielded Dose Rate Calculation [2 of 9]

Page 48

Near Side of Door Scatter Fraction Calculation					
Line	Parameter	Units	Value		Calculation
			w/o IMRT	with IMRT	
a	Max Field Size	cm	40	40	
b	Fraction of Workload		50%	50%	
c	Effective Field Area	cm ²	1600.0		$b_1 \cdot a_1^2 + b_2 \cdot a_2^2$
d	Effective Field Size	cm	40.0		$\text{sqrt}(c)$
e	Scatter Angle	deg	85		
f	Machine X-ray Energy	MV	18		
g	Scatter / 400 cm ²		2.16E-04		Function of e & f
h	Scatter Fraction		0.00086		$g \cdot c / 400$

Example 3: Direct Shielded Door Near Side of Entrance Shielded Dose Rate Calculation [3 of 9]

Page 49

Near Side of Door Shielded Dose Due to Direct Leakage

Line	Parameter	Units	Photon Leakage	Photon Scatter	Neutron Leakage	Calculation
a	Workload / Treatment	Gy/pt	3	3	3	NCRP 151 default
b	Patients per Day	pt/day	30	30	30	NCRP 151 default
c	Workload (W)	Gy/wk	450	450	450	5 * a * b
d	Use Factor	Ratio	1	1	1	
e	Fraction		1.00E-03	8.65E-04	1.50E-03	18 MV values
f	IMRT Factor		2	1	1	
g	Isocenter to Protected Point Distance	ft	21.0	21.0	21.0	
h		m	6.4	6.4	6.4	g * 0.3048
i	Unshielded Dose	mSv/wk	2.20E+01	9.50E+00	1.65E+01	1000*c*d*e*f/h^2
j	Transmission		2.88E-04	5.99E-07	1.58E-06	see below
k	Shielded Dose	mSv/wk	0.006	0.000	0.000	i * j
L	Total Shielded Dose	mSv/wk		0.006		Sum row k

Transmission Calculation for Direct Leakage at Near Side of Door

Barrier	Material Thickness		Photon Leakage			Scatter			Neutron		
	inches	mm	TVL1 (mm)	TVLc (mm)	Trans.	TVL1 (mm)	TVLc (mm)	Trans.	TVL (mm)	Trans.	
Inside Layer	48	1224	Concrete	360	349	2.88E-04	197	197	5.99E-07	211	1.58E-06
Layer #2						1.00E+00			1.00E+00		1.00E+00
Outside Layer						1.00E+00			1.00E+00		1.00E+00
Slant Angle (degrees):			5			18 MV	Total:	2.88E-04		Total:	5.99E-07
							Total:	1.00E+00		Total:	1.58E-06

Example 3: Direct Shielded Door Near Side of Entrance Shielded Dose Rate Calculation [4 of 9]

Page 50

Wall Scatter Transmission for Near Side of Door

Line	Symbol	Parameter	Units	Value	Calculation
a	MV	Machine X-ray Energy	MV	18	
b	W	Workload	Gy/wk	450	
c	f	Patient transmission		0.27	0.27 if MV ≥ 10
d	d ₀	Distance from target to primary barrier wall	ft	25	d * 0.3048
e	d ₁	Distance from primary barrier wall to near side of maze entrance	ft	9	measured
g			m	2.74	e * 0.3048
h	α ₀	Reflection coefficient	1/m ²	0.0016	Table 8a with 18 MV 85° scatter angle
i		Effective field size	cm	40.9	see above
j	A ₀	Beam area at far maze wall	m ²	9.29	(e * U/100)*2
k	U	Use Factor		0.25	Orientation with highest dose rate
L	f H _s	Wall scatter unshielded dose	mSv/wk	1.03E+00	1000 * b * c * k * h * j / (e * 2 * g * 2)

Near Side of Door Wall Scatter Transmission Calculation

Barrier	Material Thickness		Material	Wall Scatter		Photon Trans.
	inches	mm		TVL1 (mm)	TVLc (mm)	
Inside Layer	1.5	54	Lead	8	8	1.84E-07
Layer #2	5.5	198	Concrete	160	160	5.82E-02
Layer #3	3	108	Borated Poly	396	396	5.34E-01
Slant Angle (degrees):			45			0.3 MV
						Total:
						5.73E-09

Example 3: Direct Shielded Door Near Side of Entrance Shielded Dose Rate Calculation [5 of 9]

Page 51

Maze Neutron Fluence Calculation

Line	Symbol	Parameter	Units	Value	Calculation
a	MV	Machine X-ray Energy	MV	18	
b		Vendor			
c		Neutron IMRT Factor		1	
d	β	Head Transmission Factor		1	1 for lead, 0.85 for tungsten head shield
e	d ₁	Distance from isocenter to maze opening (Point A)	ft	17	measured
f			m	5.18	e * 0.3048
g	d _L	Vault Average Length	ft	28	measured
h			m	8.53	g * 0.3048
i	d _W	Vault Average Width	ft	25	measured
j			m	7.62	i * 0.3048
k	h	Vault Average Height	ft	10	measured
L			m	3.05	k * 0.3048
m	S _v	Vault Surface Area	m ²	228.5	2 * (h * j) + h * L + j * L
n	Q _n	Neutron Source Strength	n / Gy	9.60E+11	Function of a & b
o	Φ _A	Neutron Fluence at Point A per Gy	n / m ² /Gy	7.32E+09	c * n * [d / (4 * a * f * 2) + (5.4 * d + 1.3) / (2 * g * m)]

Example 3: Direct Shielded Door Near Side of Entrance Shielded Dose Rate Calculation [6 of 9]

Page 52

Capture Gamma Unshielded Dose Rate Calculation

Line	Symbol	Parameter	Units	Value	Calculation
a	MV	Machine X-ray Energy	MV	18	
b	W	Workload	Gy/wk	450	
c	Φ _A	Neutron Fluence at Point A per Gy	n / m ² /Gy	7.32E+09	see above
d	d ₂	Distance from maze opening (Point A) to door	ft	9	measured
e			m	2.74	d * 0.3048
f	TVD	Tenth-Value Distance	m	5.4	3.9 if a < 18, 5.4 otherwise
g	K	Ratio Capture Gamma Dose-Equivalent to Neutron Fluence		6.90E-16	Constant
h	h _φ	Capture Gamma Unshielded Dose at Door per Dose at Isocenter	Sv/Gy	1.57E-06	g * c * 10 ⁴ * (e / f)
i		Capture Gamma Unshielded Dose Rate	mSv/wk	7.06E-01	1000 * a * h

Example 3: Direct Shielded Door Near Side of Entrance Shielded Dose Rate Calculation [7 of 9]

Page 53

Maze Neutron Unshielded Dose-Equivalent Calculation (Modified Kersey Method)

Line	Symbol	Parameter	Units	Value	Calculation
a	W	Workload	Gy/wk	450	
b	Φ _A	Neutron Fluence at Point A per Gy	n / m ² /Gy	7.32E+09	See above
c	d ₂	Distance from maze opening (Point A) to door	ft	9	measured
d			m	2.74	c * 0.3048
e	d ₀	Inner Maze Entrance Width	ft	4	measured
f			m	1.22	e * 0.3048
g	h	Inner Maze Entrance Height	ft	10	measured
h			m	3.05	g * 0.3048
i	S ₀	Inner Maze Cross-Sectional Area	m ²	3.72	f * h
j	d _m	Maze Width	ft	4	measured
k			m	1.22	j * 0.3048
L	h _m	Average Height Along Maze	ft	10	measured
m			m	3.05	L * 0.3048
n	S	Maze Cross-Sectional Area	m ²	3.72	i * m
o	TVD _n	Maze Neutron Tenth-Value Distance	m	3.97	2.06 * sqrt(n)
p	H _{n,D}	Maze Neutron Unshielded Dose-Equivalent per Dose at Isocenter	Sv/Gy	4.62E-06	2.4E-15 * b * sqrt(i / n) * [1.84 * 10 ⁴ * (d / f) + 10 ⁴ * (d / o)]
q	H _n	Maze Neutron Unshielded Dose-Equivalent Rate at Door	mSv/wk	2.08E+00	1000 * a * p
r		Neutron Unshielded Dose-Equivalent Rate at Door with margin	mSv/wk	1.04E+01	5 * q

Example 3: Direct Shielded Door Near Side of Entrance Shielded Dose Rate Calculation [8 of 9]

Page 54

Neutron Transmission for Near Side of Maze Entrance

Barrier	Material Thickness		Material	Neutrons		Neutron Trans.
	inches	mm		TVL1 (mm)	TVLc (mm)	
Inside Layer	1.5	54	Lead	1000000	1000000	1.00E+00
Layer #2	5.5	198	Concrete	161	161	5.93E-02
Layer #3	3	108	Borated Poly	45	45	4.03E-03
Layer #4						1.00E+00
Outside Layer						1.00E+00
Slant Angle (degrees):			45			0.1 MV
						Total:
						2.39E-04

Capture Gamma Transmission for Near Side of Maze Entrance

Barrier	Material Thickness		Material	Capture Gammas		Photon Trans.
	inches	mm		TVL1 (mm)	TVLc (mm)	
Inside Layer	1.5	54	Lead	61	61	1.31E-01
Layer #2	5.5	198	Concrete	410	370	3.01E-01
Layer #3	3	108	Borated Poly	1015	916	7.63E-01
Layer #4						1.00E+00
Outside Layer						1.00E+00
Slant Angle (degrees):			45			10 MV
						Total:
						3.00E-02

Example 3: Direct Shielded Door Near Side of Entrance Shielded Dose Rate Calculation [9 of 9]

Maze Shielded Dose at Door

Line	Parameter	Units	Wall Scatter	Direct Leakage	Neutrons	Capture Gammas
a	Calc. Unshielded Dose	mSv/wk	1.03E+00	6.36E-03	1.04E+01	7.06E-01
b	Total / Calc. Dose Rate		2.64	1	1	1
c	Unshielded Dose Rate	mSv/wk	2.73E+00	6.36E-03	1.04E+01	7.06E-01
b	Energy for TVL	MV	0.3	18	0.1	10.0
c	Transmission		5.73E-09	1.00E+00	2.39E-04	3.00E-02
d	Shielded Dose	mSv/wk	0.0000	0.0064	0.0025	0.0212
e	Total Shielded Dose	mSv/wk		0.0300		

Contact Information

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